

Experimental Vertical Stability Studies for ITER Performance and Design Guidance

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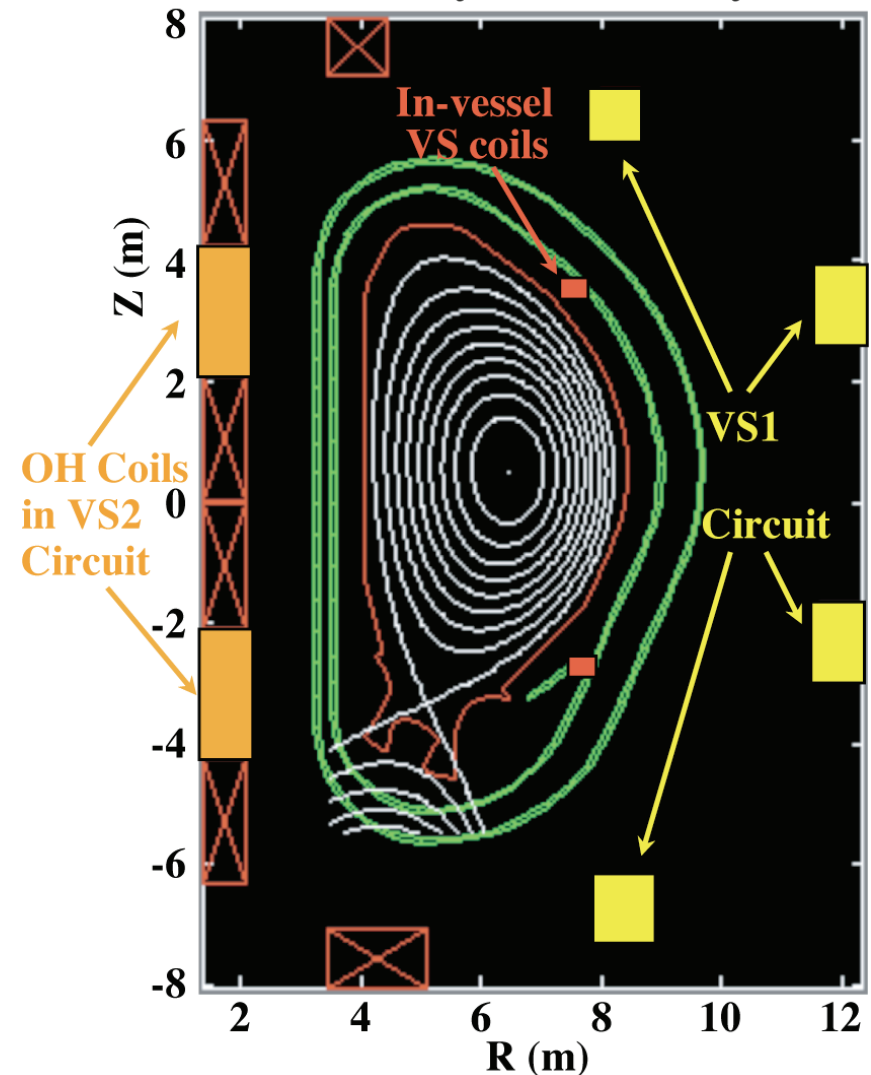
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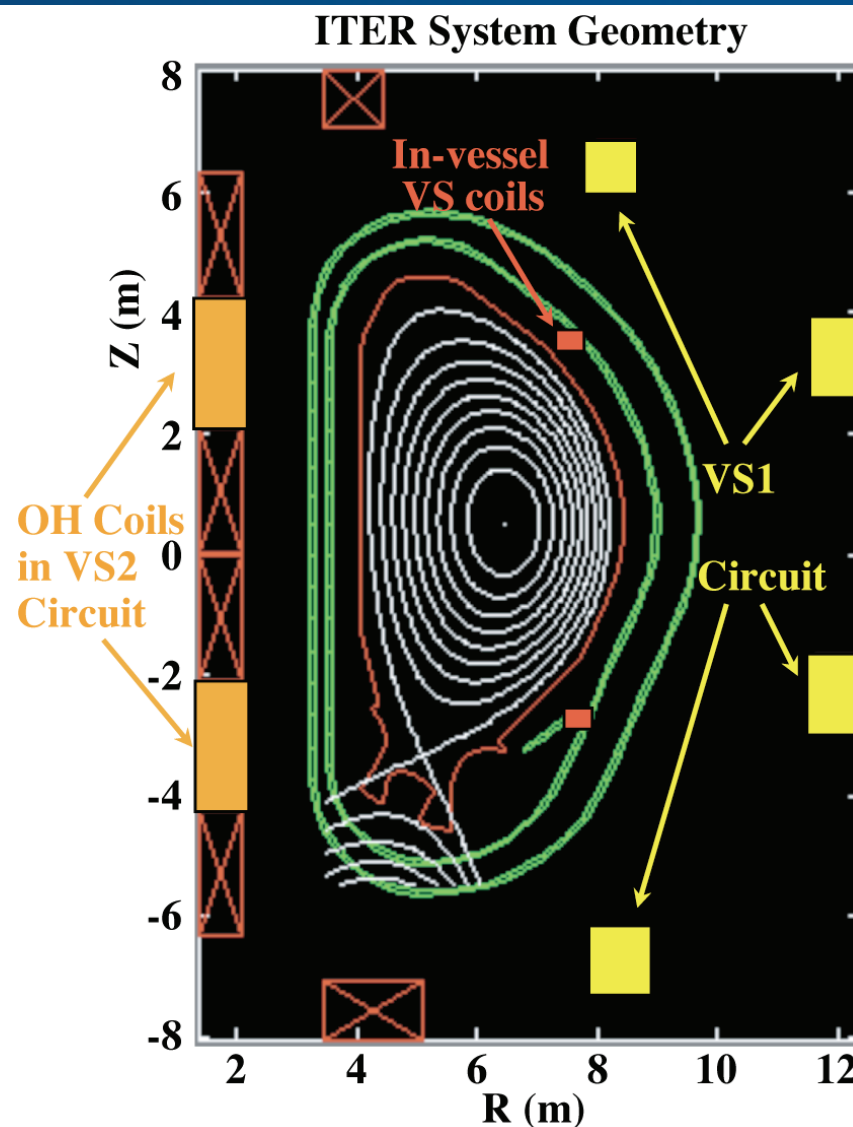
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ITER System Geometry



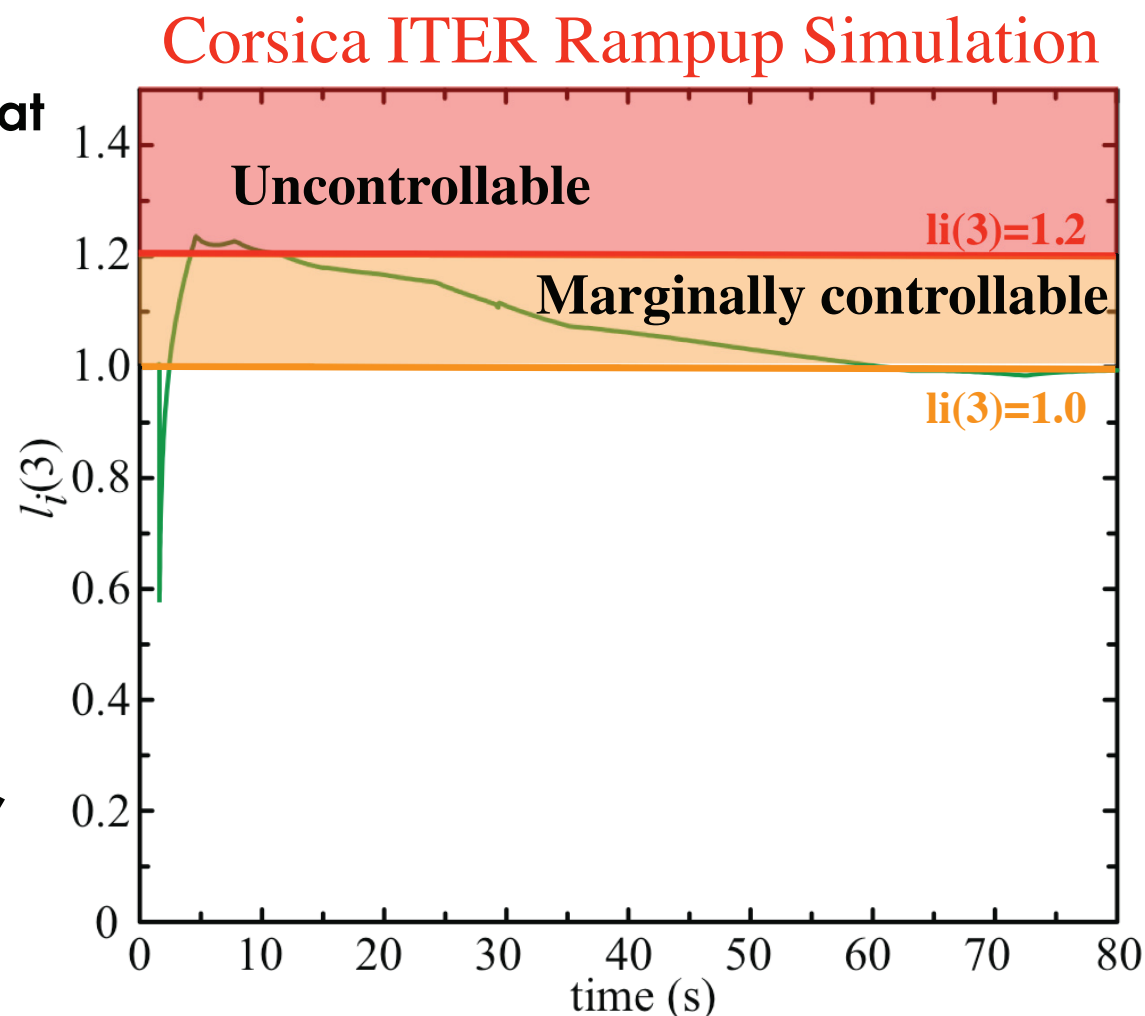
ITER Baseline System Uses Four Outboard Coils for Vertical stability Control

- Baseline system: “VS1” circuit = 4 outboard coils (PF2-5) with 6 kV operating voltage
- Proposals to enhance VS system include (from 2007-08 Design Review/STAC studies):
 - Use “VS2” circuit = 2 central solenoid coils (6 kV)
 - Add in-vessel VS coils mounted on vessel wall behind blanket modules



Baseline Scenarios Challenge Vertical Control Capability of Baseline Control System

- $1.0 < \ell_i(3) < 1.2$ during rampup at full elongation is marginally controllable by baseline VS1 system
- Uncontrollable $\ell_i(3) > 1.2$ can occur in baseline startup scenario, in high q_{95} operation, or in rampdown

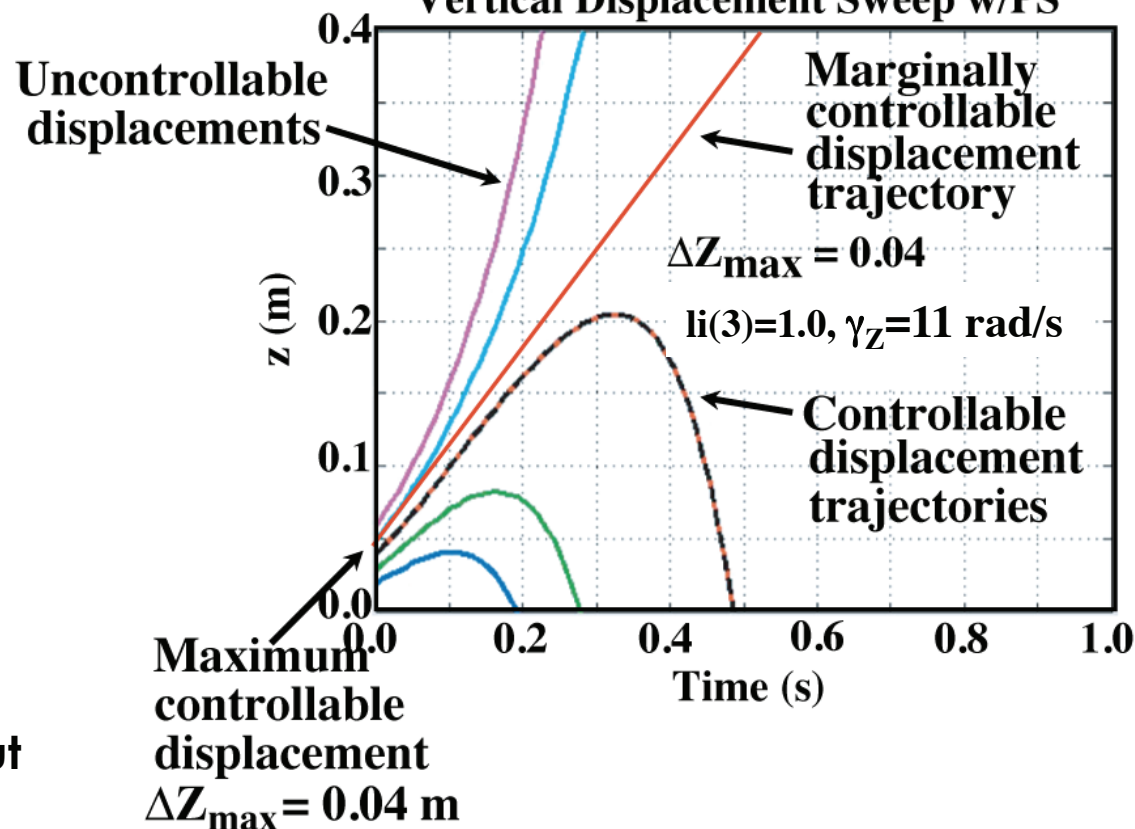


Maximum Controllable Displacement Metric Addresses Consequences of Different Voltage, Current Limits

- Plasma allowed to drift distance ΔZ
- Apply fully saturated step voltage commands to power supplies:
 - Maximum and fastest possible radial field to oppose vertical motion
- ΔZ_{MAX} = maximum value of ΔZ beyond which vertical motion cannot be reversed
- NOT a true control demonstration, but a measure of “best possible”
- $\Delta Z_{MAX}/a$ is machine-independent metric for guidance in ITER design

Example of Analysis and Gedanken Experiment to Calculate ΔZ_{max}

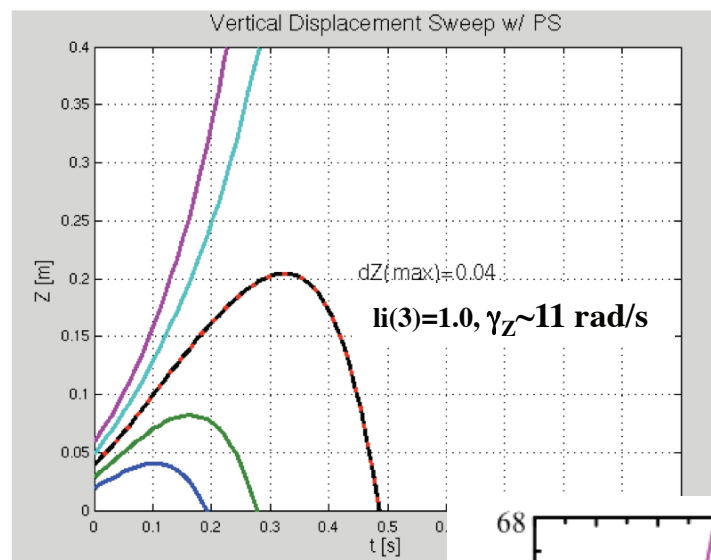
Vertical Displacement Sweep w/PS



$$\Delta Z_{max} = \frac{\partial z}{\partial I_V} \frac{M_{*VC}}{L_{*V}} \frac{V_{sat}}{L_C} \frac{1}{\gamma_z} \left(1 - e^{-\frac{\Delta I_{max} L_C \gamma_z}{V_{sat}}} \right) e^{-\gamma_z T_{PS}}$$

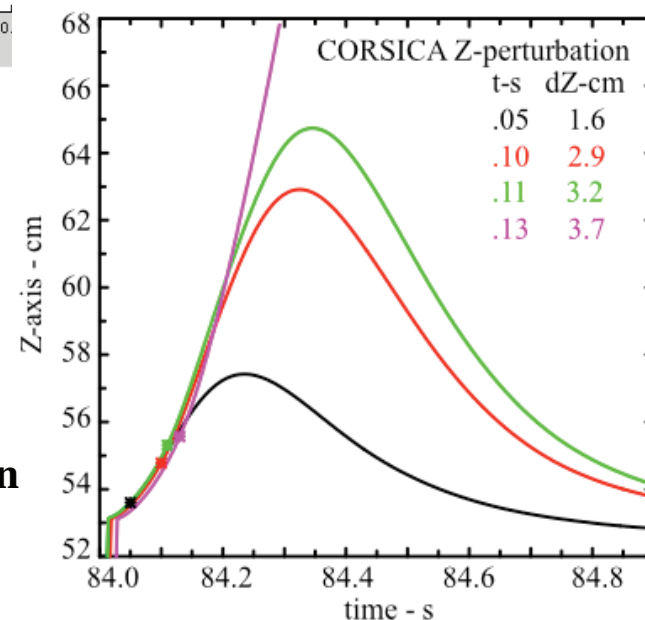
ITER Analyses Find $\Delta Z_{\max}/a \sim 2\%$ for Baseline VS1 System and End of Rampup Equilibrium

- Linear rigid TokSys model:
 $\Delta Z_{\max}/a \sim 2.1\%$
- Nonlinear nonrigid Corsica simulation: $\Delta Z_{\max}/a \sim 1.8\%$
- Question: is this capability sufficient? Need experimental guidance...

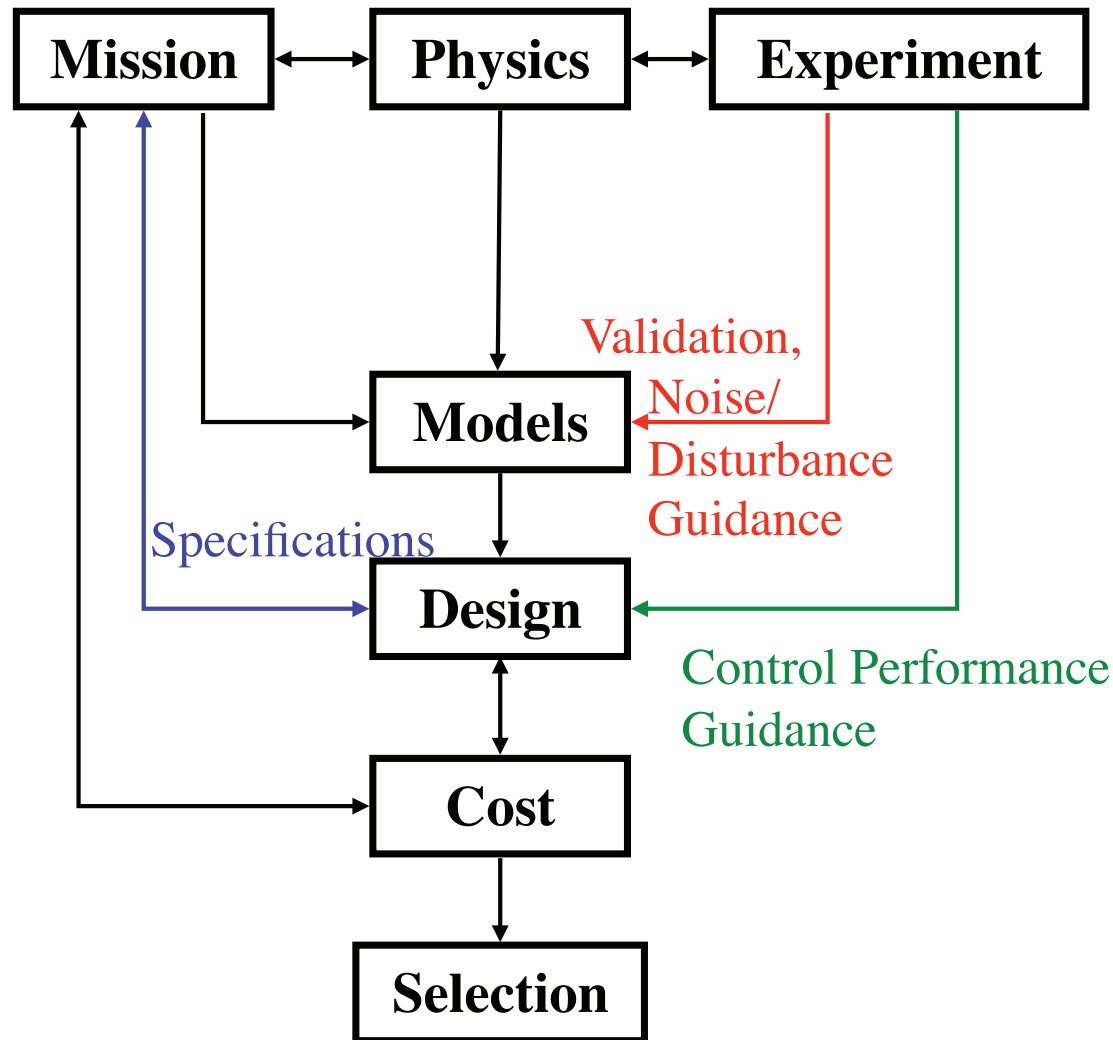


**Rigid TokSys
plasma model**

**Nonrigid Corsica
plasma simulation**



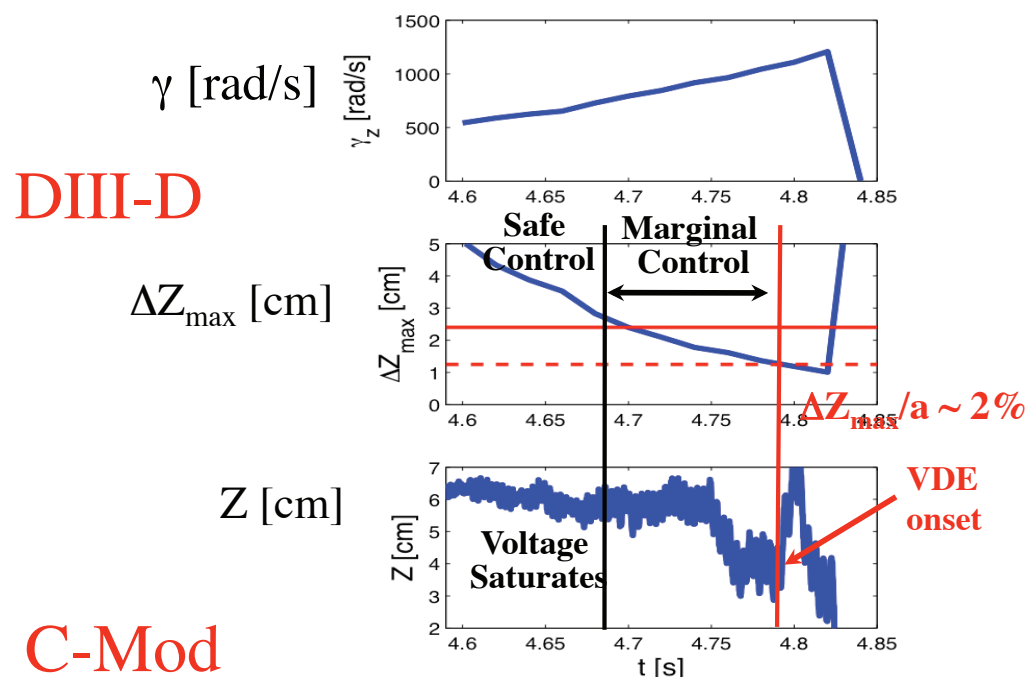
Experiments are Essential to Provide Guidance for Operational Robustness, Noise, Disturbances



- There is **no fully predictive model of noise/ disturbance environment expected in ITER:**
⇒ **Experimental guidance necessary**

Calculated $\Delta Z_{\max}/a \sim 2\%$ Guarantees Loss of Control, $\Delta Z_{\max}/a \sim 4\%$ Marginal in DIII-D and C-Mod

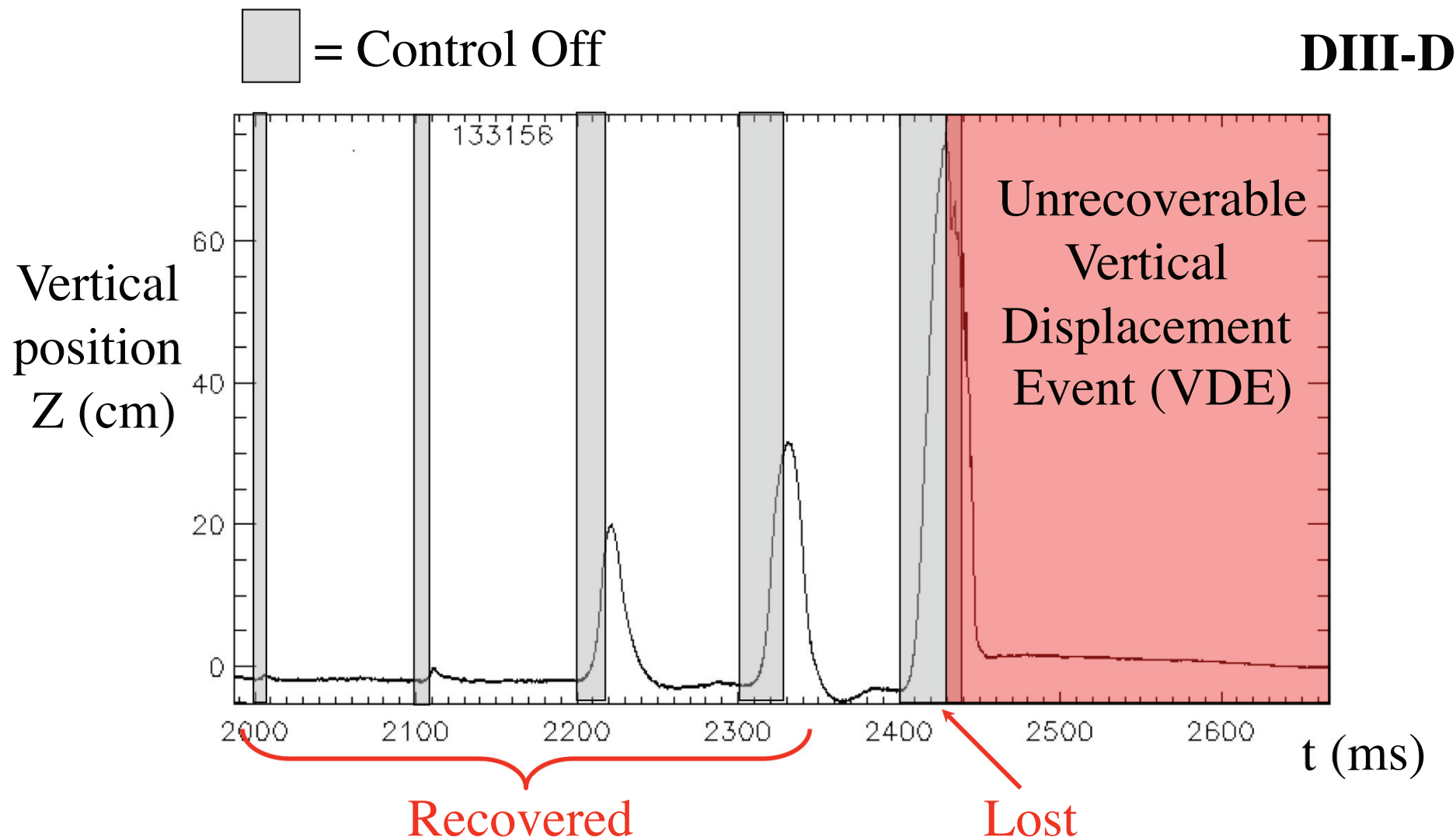
- Experiments performed in DIII-D and C-Mod changing elongation (growth rate) to find limit to vertical control
- $\Delta Z_{\max}/a \sim 2\%$ guarantees loss of vertical control
- Marginal $\Delta Z_{\max}$ in both machines corresponds to $\Delta Z_{\max}/a \sim 4\%$
- “Safe” operation in both machines corresponds to $\Delta Z_{\max}/a > 5\%$
- Typical robust operation corresponds to $\Delta Z_{\max}/a > 10\%$



Case	γ_z (rad/s)	m_s	$\Delta Z_{\max}$ (cm)	$\Delta Z_{\max}/a$ (%)	$\Delta Z_{\max}/\langle \Delta Z_{\text{noise}} \rangle$
1	210	0.41	2.8	13%	28
2	260	0.37	2.1	9.7%	21
3	310	0.33	1.5	6.9%	15
4	410	0.28	0.8	3.7%	8

Unsafe C-Mod operating point

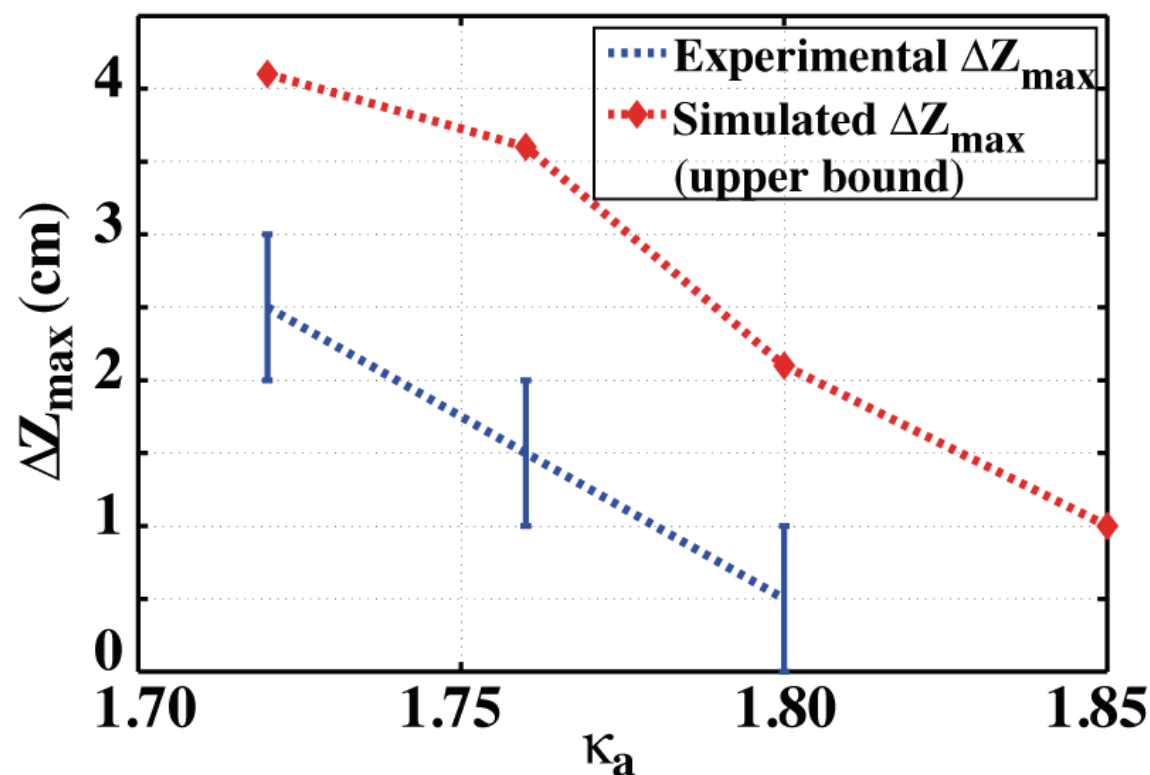
Experiments Varying the Control Off Time and Vertical Displacement Search Large ΔZ Space in One Shot



Alcator C-Mod $\Delta Z_{\max}$ Experiment Shows Predicted Values Are About Twice Experimental Values

- Elongation K_a varied, vertical control disabled for varying periods
- **Calculated $\Delta Z_{\max} \sim 30\text{-}50\%$ above maximum experimental value for each K_a**
- Alcator C-Mod $\Delta Z_{\max}$ determined by coil current limit, not voltage limit
 - Similar to proposed ITER in-vessel coils

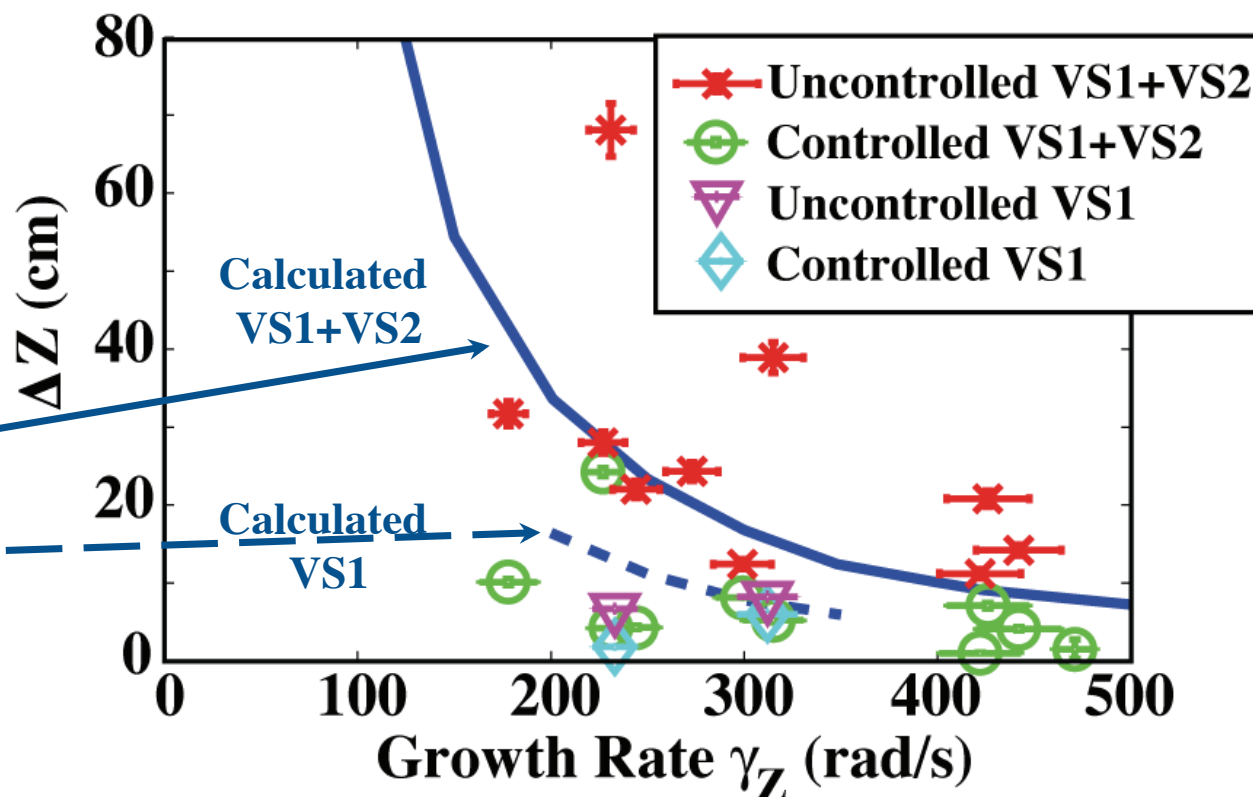
Confidence Intervals for $\Delta Z_{\max}$ Measurements on Alcator C-Mod



DIII-D $\Delta Z_{\max}$ Experiment Shows Use of Inboard Coils Approximately Doubles Performance

- Elongation varied to vary growth rate in different discharges
- Use of **inboard+outboard** coils ~ doubles $\Delta Z_{\max}$ of **outboard-only**
- **Calculated $\Delta Z_{\max}$ ~20-30% above experimental values**

DIII-D $\Delta Z_{\max}$ Experimental Summary

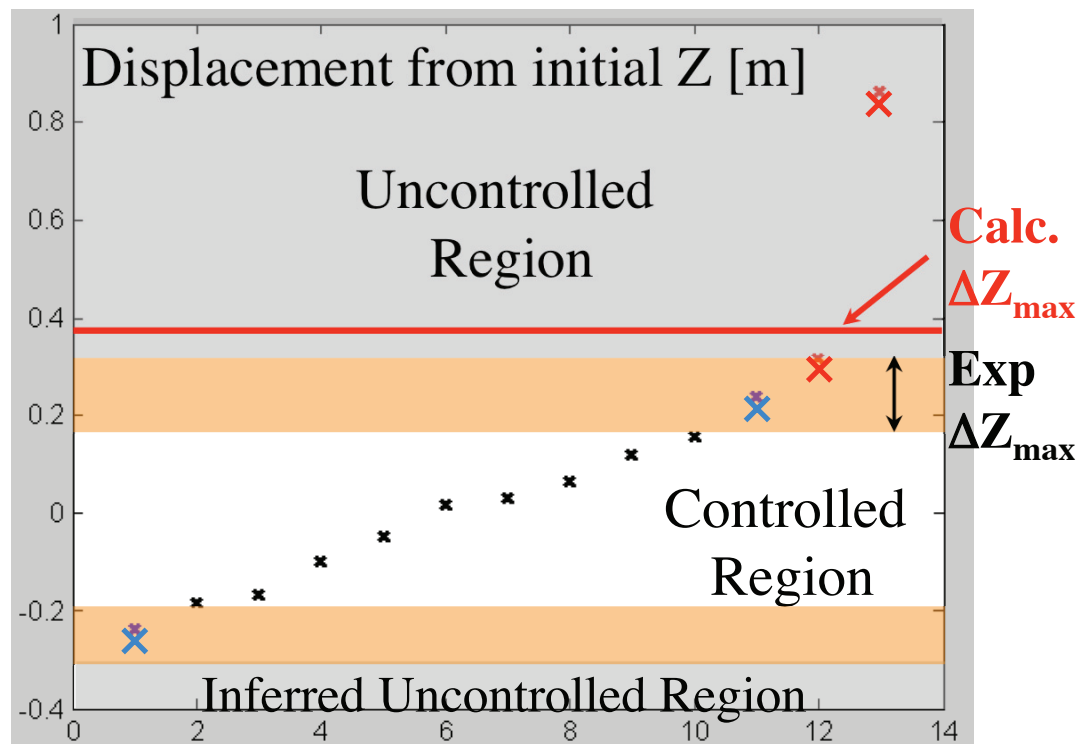


VS1 = outboard coilset

VS2 = inboard coilset

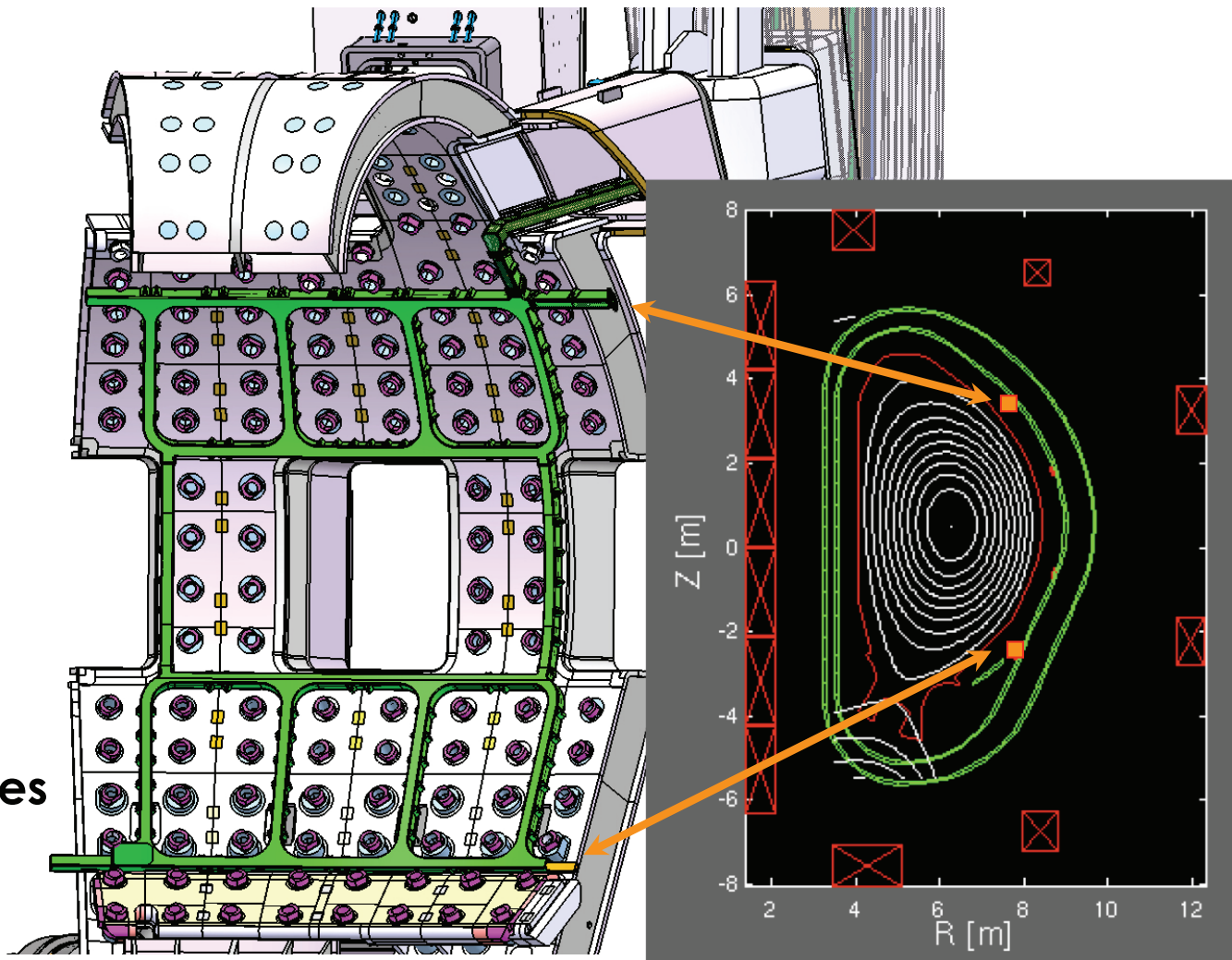
NSTX $\Delta Z_{\max}$ Experiment Shows Predicted Value ~30% Greater than Experiment

- Single equilibrium target was studied with varying distances of vertical drift:
 - Single growth rate and $\Delta Z_{\max}$ value
 - Finely resolved range of ΔZ cases
- Calculated value $\Delta Z_{\max} \sim 30\%$ above experimental maximum
- Potential role of nonaxisymmetric conductors, nonlinear interaction with limiter
 - Suggests need for modeling 3D effects of ITER conductors



ITER Design Now Incorporates In-Vessel Coils for Increased Vertical Control Capability

- In-vessel coils alone provide $\Delta Z_{\max}/a \sim 5\%$ capability
- Pose significant engineering design, fabrication, and maintenance challenges



Summary and Conclusions

- **Multi-machine experiments for vertical control performance have:**
 - Quantified vertical control performance in present devices
 - Partially validated theoretical performance scalings
 - Translated performance data/analysis into metric specifications
- **Experiments/analysis have provided key motivation for improving ITER vertical control capability:**
 - $\Delta Z_{\max}/a > 5\%$ required for robust control at edge of ITER operating space
 - $\Delta Z_{\max}/a \sim 2\%$ is capability of ITER baseline system
 - ITER in-vessel coils being designed to provide $\Delta Z_{\max}/a > 5\%$
 - Discrepancies between calculation and experiment emphasize need for margin in design
- **Work remains to carefully analyze physics of control limits:**
 - Details of noise-determined control limits
 - Types, magnitudes, detailed histories of disturbances to expect in ITER

Multi-machine Noise Data Shows $\langle Z \rangle_{\text{noise}}/a \sim 0.5\text{--}1\%$

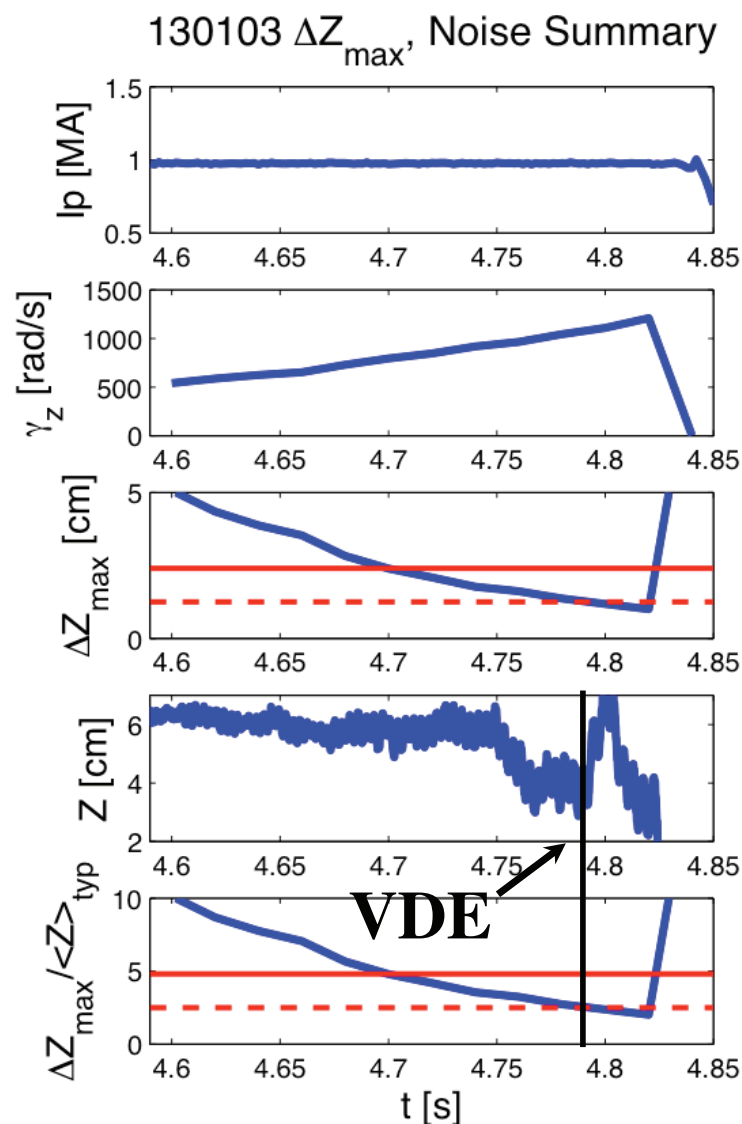
- Data represents all sources of noise in real-time vertical position estimator for each device ($\langle Z \rangle_{\text{noise}}$ = standard deviation in “typical” operation)
- Noise contributions include instrumentation, power supply pickup, discrete measurement reconstruction error, plasma instability-driven signals, etc.
- TCV underwent extensive noise-abatement process to reduce $\langle Z \rangle_{\text{noise}}$
- Implies $\Delta Z_{\text{max}}/a \sim 5\%$ corresponds to $\Delta Z_{\text{max}}/\langle Z \rangle_{\text{noise}} \sim 5\text{--}10$ in present experiments

Device	Typical $\langle Z \rangle_{\text{rms}}$ (cm)	Minor radius, a (cm)	$\langle Z \rangle/a$ (%)
Alcator C-Mod	0.10	21	0.5
DIII-D	0.4	60	0.7
JET	1.4	100	1.4
NSTX	0.7	63	1.1
TCV	0.05	25	0.2

Controllability Threshold Experiments in DIII-D Show

$$\Delta Z_{\max}/\langle Z \rangle_{\text{noise}} \sim 2-3 \text{ Assures VDE}$$

- DIII-D: increasing elongation in single discharge to increase growth rate γ_z until VDE
- Uncontrollable VDE occurs at $\Delta Z_{\max}/\langle Z \rangle_{\text{noise}} \sim 2-3$
- Consistent with $\Delta Z_{\max}/\langle Z \rangle_{\text{noise}}$ controllability threshold data in DIII-D:
 - VDE guaranteed at $\Delta Z_{\max}/a \sim 2\%$
 - Typical $\langle Z \rangle_{\text{noise}}/a \sim 0.7\%$
 - $\Delta Z_{\max}/\langle Z \rangle_{\text{noise}} \sim 3$ assures VDE
- Marginal control in Alcator C-Mod and DIII-D corresponds to:
 - $\Delta Z_{\max}/a \sim 4\%$
 - $\Delta Z_{\max}/\langle Z \rangle_{\text{noise}} \sim 5-8$



Disturbances Alone Will Require ITER to Have Sufficient $\Delta Z_{\max}/a$ Capability

- Noise sets typical control limits in DIII-D, Alcator C-Mod, JET, NSTX:
 - Noise-driven $\langle Z \rangle_{\text{RMS}}/a \sim 0.5-1\%$ in these
- It is possible that ITER will have lower noise than US machines:
 - CRPP-Lausanne has contract for magnetic diagnostics/noise reduction
 - Noise reduction procedure demonstrated in TCV: $\langle Z \rangle_{\text{RMS}}/a < 0.2\%$
- Nevertheless, various plasma disturbances may set limit to control in ITER:
 - DIII-D ITER demonstration discharges produce ELM-driven plasma perturbation $\Delta Z_{\text{ELM}}/a \sim 3\%$

